

**Special Instructions:**

1. Closed Book. All calculators and one 8½ x 11 inch crib sheet (both sides) are permitted.
2. Candidates must attempt all questions.
3. The value of each question is as indicated.
4. Point form is acceptable for discussion-type questions.

**TOTAL MARKS: 105**

**THIS EXAMINATION PAPER INCLUDES 3 PAGES AND 10 QUESTIONS. YOU ARE RESPONSIBLE FOR ENSURING THAT YOUR COPY OF THE PAPER IS COMPLETE. BRING ANY DISCREPANCY TO THE ATTENTION OF YOUR INVIGILATOR.**

1. (2 marks each) Define, in words:
  - a.  $\beta$ , in the neutron sense
  - b. Neutron current
  - c. Macroscopic cross-section
  - d. Reactivity coefficient
  - e. Prompt critical
  - f. Delayed neutron
  - g. Diffusion length
  - h. Moderation
  - i. Extrapolation distance
  - j. Neutron flux.

2. (12 marks) Consider the neutron continuity equation:

$$\frac{1}{v} \frac{\partial \phi(\mathbf{P}, E, t)}{\partial t} + \nabla \cdot \mathbf{J}(\mathbf{P}, E, t) - \Sigma(\mathbf{P}, E, t) \phi(\mathbf{P}, E, t) = \chi_m \int_0^4 v \Sigma_f(\mathbf{P}, E', t) \phi(\mathbf{P}, E', t) dE' - \sum_{g=1}^4 \Sigma_s(\mathbf{P}, E) \phi_g(\mathbf{P}, E, t)$$

We have discussed many approximations to this equation. Outline but do not derive them.

3. (5 marks) Consider the equation

$$\nabla \cdot \mathbf{J}_g \phi_g = \sum_g \chi_g \phi_g + \sum_{j=1}^G v_j \Sigma_{f(j)} \phi_j - \sum_{j=1}^G \Sigma_{(j)g} \phi_j$$

What physical process is represented by each term? Do not define individual symbols.

4. (10 marks) Consider the point kinetics “power” equation:

$$\frac{dP}{dt} = \left( \frac{\rho - \beta}{\Lambda} \right) P + \sum_{j=1}^6 \lambda_j C_j$$

Discuss how the terms on the right-hand side affect the time behaviour of the reactor power when a reactivity change takes place, particularly when the magnitude of the effect is compared to  $\beta$ , the delayed neutron fraction.

5. (7 marks) Develop finite-difference equivalents for the following equations:

$$\frac{dP}{dt} = \left( \frac{\rho - \beta}{\Lambda} \right) P + \sum_{j=1}^6 \lambda_j C_j \quad \frac{dC_j}{dt} = -\frac{\beta_j}{\Lambda} P + \lambda_j C_j ; j = 1, 6$$

Which quantities affect the implementation of the numerical method? Why?

6. Consider a neutron interacting with U-235.

- (6 marks) Draw a sketch showing the possible processes and particles.
- (12 marks) Write down appropriate differential equations which describe the time behaviour of the populations of particles associated with fission.

7. (8 marks) Use the appropriate rate equations to develop an expression for the steady-state concentration of xenon-135. Show that this quantity has a limiting value as flux increases.

8. A reactor is operating at a steady power of 120 MW when it is tripped by its emergency shutdown system, which has a total reactivity worth of -80 milli-k. Assume a U-235 reactor with one delayed neutron group such that  $\beta = 0.0065$  and  $\lambda = 0.08 \text{ s}^{-1}$ .

- (4 marks) Determine the power immediately after the trip.
- (4 marks) How long does it take for the power to reach 1% of the steady-state operating power?

9. (10 marks) An infinitely-long and  $z$ -thick bare slab of width  $2w$  consists of a fissile, absorbing material. It also contains a distributed neutron source of the form

$$S(x) = S_0 e^{-\alpha|x|}, \quad -w \leq x \leq w$$

where  $\alpha$  is a constant. The slab is bounded by vacuum. Assume all material properties are independent of position. Ignore any explicit consideration of the extrapolation distance. Develop an expression for the flux as a function of position. Work through the problem solution to the point where you need to determine the expressions for the unknown constants in the homogeneous solution of the differential equation; then, outline only how you would calculate these constants.

10. (7 marks) A reactor is operating at a steady-state power  $P$ . The measured reactor power is given by

$$P = c_p \rho (T_{outlet} - T_{inlet})$$

where  $T_{inlet}$  is fixed. A small positive step reactivity perturbation of magnitude  $\rho_0$  occurs. Assume a single temperature coefficient  $\alpha$  is an adequate model for the behaviour. Estimate the power at which temperature feedback stops the power increase.

*Hint: use an average temperature to represent the core's thermal behaviour.*

**THE END**